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### RESEARCH ARTICLES

- 1901** Neutron Total Cross Sections of  $^{232}\text{Th}$ ,  $^{237}\text{Np}$ ,  $^{235,238}\text{U}$ , and  $^{239}\text{Pu}$  from 3 to 230 MeV  
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- 1935** Evaluation of Kinetic Parameters RSG-GAS Reactor Equilibrium Silicide Core Using Continuous-Energy Monte Carlo Serpent 2 Code  
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- 1950** Calculation of Thermalized Component of Neutron Spectra in a D-D Neutron Generator  
*Nnaemeka Nnamani*
- 1958** Development of HELIOS/BIPR/PARCS/MCNP6 Computation Route for WWER RPV Neutron Fluence Analysis and Validation Against Ex-Vessel Detector Measurement Data  
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- 1965** CFD Analysis of Effects of Grid Spacer Vane on Thermal-Hydraulic Performance of Fluid in a 5×5 Fuel Channel  
*Satish Kumar Dhurandhar, S. L. Sinha, Shashi Kant Verma*
- 1984** Numerical Simulation of Turbulent Flow and Friction Characteristics Through a Loop of Narrow Rectangular Channel Under Rolling Motions  
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- 1998** Feasibility of Using Mixed-Oxide Fuel for a Pressurized Water Reactor Instead of Traditional  $\text{UO}_2$  Fuel Material  
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*Arief Rahman Hakim, Douglas A. Fynan*

### NOTE

- 2038** Quantifying Safety Significance: An In-Depth Analysis of Importance Measures in Level 1 PSA for the VVR 10-MW Water-Water Research Reactor  
*Rex Gyeabour Abrefah, Felix Ameyaw*