

nuclear technology®

CONTENTS / NOVEMBER 1995—VOL. 112, NO. 2

TECHNICAL PAPERS

NUCLEAR REACTOR SAFETY

- 155 Mitigation of an Anticipated Transient Without Scram Event in a Simplified Boiling Water Reactor by the Insertion of Fine-Motion Control Rods / *Hasna J. Khan, Hsiang S. Cheng, Upendra S. Rohatgi*
- 169 Independent Assessment of MELCOR as a Severe Accident Thermal-Hydraulic/Source Term Analysis Tool / *I. K. Madni*
- 181 A Comparative Simulation of Feed and Bleed Operation During the Total Loss of Feedwater Event by RELAP5/MOD3 and CEFLASH-4AS/REM Computer Codes / *Young Min Kwon, Tae Sun Ro, Jin Ho Song*

NUCLEAR FUEL CYCLES

- 194 Irradiation Performance of Fast Flux Test Facility Drivers Using D9 Alloy / *A. L. Pitner, B. C. Gneiting, F. E. Bard*

RADIOISOTOPES AND ISOTOPES

- 204 Choice of a Process Design for Simultaneous Detritiation and Upgrading of Heavy Water for the Advanced Neutron Source / *A. I. Miller, D. A. Spagnolo, J. R. DeVore*

HEAT TRANSFER AND FLUID FLOW

- 214 Taiwan Power Company's Power Distribution Analysis and Fuel Thermal Margin Verification Methods for Pressurized Water Reactors / *Ping-Hue Huang*
- 227 Heat Transfer from a Plate Cooled by a Water Film with Countercurrent Air Flow / *W. Ambrosini, A. Manfredini, F. Mariotti, F. Oriolo, P. Vigni*
- 238 Thermal-Hydraulic Modeling and Severe Accident Radionuclide Transport / *F. Oriolo, W. Ambrosini, G. Fruttuoso, F. Parozzi, R. Fontana*

REACTOR OPERATIONS

- 250 Probabilistic Analysis of Limiting Conditions for Operation Action Requirements Including Risk of Shutdown / *T. Mankamo, I. S. Kim, P. K. Samanta*

REACTOR CONTROL

- 266 Development of the On-Line Operator Aid System OASYS Using a Rule-Based Expert System and Fuzzy Logic for Nuclear Power Plants / *Soon Heung Chang, Ki Sig Kang, Seong Soo Choi, Han Gon Kim, Hee Kyo Jeong, Chul Un Yi*

TECHNICAL NOTE

NUCLEAR FUEL CYCLES

- 295 Transient Gap Conductance in Nuclear Fuel Rods / *G. Dharmadurai*