

13. Other Programming and Operating Information or Restrictions: The program is compiled by XFAM 4E and listed by editor XMUM MARK 3A.
14. Material Available: A source deck, sample problem, and operating instructions are available from the authors.
15. *Acknowledgment*: This paper is based on work performed under State Committee for Nuclear Energy Contract 134/7 (1970).
16. *References*:
 M. PAVELESCU, V. ANTON, I. PURICA, "Numerical Method for Solving Transport Equation in the Case of Multizone Nuclear Reactors," Preprint IFA FR-85-1971, Institutt for Atomeneigi, Kjeller, Norway.
 I. PURICA, "Optimization of the Initial Breeding Ratio of a Nuclear Reactor Considered as a Game Theory Problem," *Proc. U. N. Intern. Conf. Peaceful Uses At. Energy, 3rd*, Geneva, P/674 (1964).
 "Reactor Physics Constants," ANL-5800, 2nd ed., Argonne National Laboratory (1963).
4. Method of Solution: SNG.
5. Restrictions on the Complexity of the Problem: 50 spatial points, 16 energy groups, 5 angular directions, 5 nuclides, 3 zones of heterogeneity.
6. Typical Running Time: 30 min.
7. Unusual Features of the Program: Number of spatial points used in a zone is proportional to its thickness.
8. Related and Auxiliary Programs: The macroscopic multigroup cross sections are computed in the code itself; therefore, no relation with auxiliary program is required.
9. Status: In Use.
10. Machine Requirements: 28K words core and no peripheral storage device are required.
11. Programming Language Used: FORTRAN-1900.
12. Operating System or Monitor Under Which Program is Executed: Supervisor and, in particular, George 2.
13. Other Programming and Operating Information or Restrictions: The program is compiled by XFAM 4E and listed by editor XMUM MARK 3A.
14. Material Available: A source deck, sample problem, and operating instructions are available from the authors.
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16. *References*:
 M. PAVELESCU, V. ANTON, and I. PURICA, "Numerical Method SNG," Preprint IFA, FR-81-1971, Institutt for Atomenergi, Kjeller.
 "Reactor Physics Constants," ANL-5800, 2nd ed., Argonne National Laboratory (1963).

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Received April 7, 1971
Revised August 16, 1971

APEL

1. Name of Code: APEL.
2. Computer for Which Program is Designed: ICL-1905.
3. Nature of Physical Problem Solved: APEL solves the time-independent neutron transport approximation for spatially dependent neutron flux distribution in a spherical geometry. The code provides critical parameters (flux distribution, critical mass, reaction rates, initial breeding ratio) for given volume fractions of the fast system.

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