

In a  $P_N$  approximation, one has to take the moments of the balance Eq. (1) in the half-space. The integration is either over all the  $\Omega$  directed into the medium at  $r$ , or over all the surface  $S'$  seen from  $dS$  at  $r$ .

Now, even-order and odd-order spherical harmonics represent a system of orthogonal functions in the half-space. The decision for the even-order spherical harmonics arises from the argument to retain a special balance equation for the current density. This equation follows from Eq. (1) as the zero-moment with  $(1, m) = (0, 0)$ . The cavity boundary conditions in a  $P_N$  approximation, therefore, take the form

$$\int_{\Omega \cdot n < 0} Y_l^m \Phi(r, \Omega) (\Omega \cdot n) d\Omega = \int_{\Omega \cdot n < 0} Y_l^m \Phi(r', \Omega) (\Omega \cdot n) d\Omega$$

$$= \int_{S'} Y_l^m \Phi(r', \Omega) \frac{(\Omega \cdot n)(\Omega \cdot n')}{R^2} dS',$$

$$l = 0, 2, 4, \dots, \quad N-1, \quad N \text{ odd}, \quad -1 \leq m \leq 1. \quad (3)$$

For zero exposure, Eq. (3) reduces to the vacuum boundary conditions

$$\int_{\Omega \cdot n < 0} Y_l^m \Phi(r, \Omega) (\Omega \cdot n) d\Omega = 0$$

$$l = 0, 2, 4, \dots, \quad N-1, \quad N \text{ odd}, \quad -1 \leq m \leq 1. \quad (4)$$

In contrast to the generalized Marshak-type boundary conditions cited by Davison<sup>3</sup>, Eqs. (3) and (4) represent just the right number of cavity and vacuum boundary conditions for more-dimensional problems.

D. Emendörfer

Institut für Kernenergetik  
Technische Hochschule  
Stuttgart, Germany

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<sup>3</sup>B. DAVISON, *Neutron Transport Theory*, Clarendon Press, Oxford, N. Y. (1957).

### Fine Group Cross Sections of Resonance Absorbers

Previously<sup>1</sup> a definition of multigroup resonance cross section was given as

$$\Sigma_i = \Sigma_i^* C_i,$$

where

$$\Sigma_i^* = (1 - p_i) \bar{\xi} \Sigma_p / p_i \delta u$$

and

$$C_i = 1 + (\Sigma_g - \Sigma_i^*) \delta u / 2 \bar{\xi} \Sigma_p. \quad (1)$$

Here,  $i$  refers to the  $i$ 'th resonance of a group  $g$  of width  $\delta u$  lethargy units which contains  $n$  resonances

$\bar{\xi}$  = average lethargy gain on collision

$\Sigma_p$  = potential scattering cross section

$$\Sigma_g = \sum_{i=1}^n \Sigma_i^*$$

$p_i$  = resonance escape probability.

For a fine group ( $\delta u < 2\bar{\xi}$  approximately), the previous derivation needs to be extended to include the possibility of neutrons skipping the group. Consider a group below source energies for a source emitting  $1n/(cm^3 \text{ sec})$  and above thermal energies so that slowing down theory applies. Neutron balance for an infinite homogeneous system gives the equation

$$\sum_{i=1}^n \Sigma_i \phi_g + \Sigma_r \phi_g = f, \quad (2)$$

and we will assume that higher groups do not contain absorption. In Eq. (2),  $\phi_g$  is the group flux,  $\Sigma_r$  is the group removal cross section, and  $f$  is the probability of a neutron entering the group given by

$$f = \Sigma_r \delta u / \bar{\xi} \Sigma_p$$

$$= \sum_l \Sigma_{pl} \xi_l f_l / \bar{\xi} \Sigma_p. \quad (3)$$

Here, the summation on  $l$  is taken over all nuclides with potential scattering cross section  $\Sigma_{pl}$  and average lethargy gain on collision  $\xi_l$  and

$$f_l = \left[ \begin{array}{l} 1 - \exp(-\delta u) - \alpha_l \delta u / \xi_l (1 - \alpha_l), \quad \delta u < \ln^1 / \alpha_l (\cong 2\xi_l) \\ = 1, \quad \delta u \geq \ln^1 / \alpha_l \end{array} \right], \quad (4)$$

where  $\alpha_l$  is the maximum fractional energy loss on collision with nuclide  $l$ .

To obtain the correct reaction rates we must have

$$\Sigma_i \phi_g = \prod_{k=0}^{i-1} p_k (1 - p_i), \quad \text{with } p_0 = 1, \quad (5)$$

provided we assume the flux to return to asymptotic form between resonances. From Eqs. (2), (3), and (5) we have

$$\phi_g = \left( \prod_{k=1}^n p_k + f - 1 \right) \delta u / \bar{f} \bar{\xi} \Sigma_p, \quad (6)$$

and from Eqs. (5) and (6)

$$\Sigma_i = \Sigma_i^* C_i,$$

where  $\Sigma_i^*$  is given by Eq. (1), but now

$$C_i = \prod_{k=1}^i p_k / \left[ 1 - \left( 1 - \prod_{k=1}^n p_k \right) / f \right], \quad (7)$$

which must be simplified.

We require an approximation to  $C_i$  that preserves neutron balance given by Eq. (2) but which does not distinguish between the actual ordering or resonances in the group. We assume that

$$C_i \approx 1 + a \left( \sum_{k=1}^n \epsilon_k - \epsilon_i \right) + b \epsilon_i, \quad (8)$$

where  $\epsilon_i = 1 - p_i$ . Equation (2) gives

$$\sum_{i=1}^n \epsilon_i \left[ 1 + a \left( \sum_{k=1}^n \epsilon_k - \epsilon_i \right) + b \epsilon_i \right] /$$

$$f(1 - \epsilon_i) \approx \left\{ 1 - \left[ 1 - \prod_{k=1}^n (1 - \epsilon_k) \right] / f \right\}^{-1} - 1, \quad (9)$$

and we require this to be accurate to terms  $\epsilon_i \epsilon_k$ . When only one resonance is in the group, we require

$$b = \frac{1}{f} - 1.$$

<sup>1</sup>A. KEANE and J. P. POLLARD, *Nucl. Sci. Eng.*, 25, 4, 439 (1966).

Using this choice in Eq. (8) then gives

$$a = \frac{1}{f} - \frac{1}{2} .$$

Equation (8) thus becomes

$$C_i \simeq 1 + \left[ \left( \frac{2}{f} - 1 \right) \Sigma_g - \Sigma_i^* \right] \delta u / 2 \bar{\xi} \Sigma_p . \quad (10)$$

For fine groups, Eq. (10) should replace the corresponding Eq. (1). As the width of the group is increased, both equations become identical.

*J. P. Pollard*

Australian Atomic Energy Commission  
Lucas Heights, N. S. W.  
Australia

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